

# Preliminary Design Study of a Fast-Ramping Magnet for Preconcept Design of an Electron–Ion Collider at Jefferson Lab

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**Abstract**—The Jefferson Lab electron–ion collider is a proposed new machine for nuclear physics research. The all new ion accelerator and collider complex will consist of two collider rings with a unique figure-of-eight layout to deliver a high degree of polarization in both beams. As a part of the preconcept design for the Ion ring system, a fast ramping 3-T superferric dipole magnet was proposed utilizing a superconducting cable-in-conduit-conductor (CICC) design to wind the coils which will be built by Texas A&M University. The first mechanical model of the winding for the 3T-SF-CICC dipole was built to validate that the winding structure provides the conductor geometry required to provide the collider field homogeneity over large aperture. A rapid-cycling Booster synchrotron is required to inject 8 GeV beams to the Ion Ring. The Booster requires arc dipoles with the same field and aperture as those of the Ion Ring. Due to the design of the CICC with respect to the amount of stabilizer and the internal cooling mechanism employed, temperature rise of the coils during a quench event is much more rapid than for more conventional magnets. It is, thus, imperative that the magnet's stored energy is dissipated externally to the windings to avoid overheating the CICC and to provide adequate protection during a quench. This article presents a preliminary design study, including ac effects inside the coils and associated risks with the aim of providing guidance for the design of the full protection system for such a magnet.

**Index Terms**—Cable-in-conduit-conductor (CICC), Jefferson lab electron–ion collider (JLEIC), quench protection, super-ferric dipole.

## I. INTRODUCTION

MODERN nuclear physics has led to the development of quantum chromo-dynamics (QCD), a theory describing strong interactions among quarks and leptons, and

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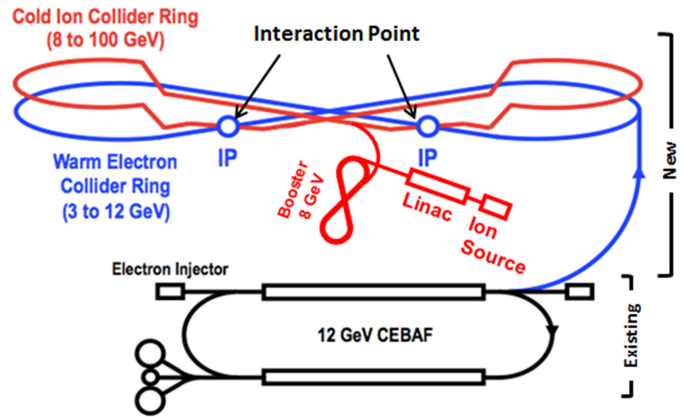


Fig. 1. Schematic drawing of the JLEIC at the JLab site.

gluons as the carriers of the strong force. A polarized electron–ion collider (EIC) has long been envisioned as a gluon microscope for exploring the QCD frontier. To meet this need, Jefferson Lab (JLab) has proposed the Jefferson Lab electron–ion collider (JLEIC), a high-luminosity high-polarization EIC based on the existing continuous electron beam accelerator facility electron superconducting radiofrequency (SRF) Linac. The original preconcept proposal (for which the 3-T magnet was designed) was to deliver between 15 and 65 GeV center of mass energy collisions between electrons and ions. The ion accelerator and collider complex would deliver up to 100 GeV protons or 40 GeV/nucleon ions. JLEIC will be a ring–ring collider, namely, both colliding electron and ion beams are stored in two figure-8 shaped rings as shown in Fig. 1 [1]. The JLEIC design was driven by the science program summarized in an EIC white paper [2]. Many significant design studies and accelerator R&D pursued over the years have concentrated on the following key topics—ion ring magnets, interaction region design, and special magnets, e.g., the booster magnet.

A 3T Super-Ferric (3T-SF) dipole magnet was designed by the group at Texas A&M University (TAMU), Accelerator Research Lab [3] for the requirements of the lattice of the JLEIC Ion Ring: 4 m body length, large aperture ( $10 \times 6 \text{ cm}^2$ ) with homogeneous field, and modest ramp rate capability (0.1 T/s). The design utilizes a novel variant of superconducting cable-in-conduit

TABLE I  
SYMBOLS AND ABBREVIATIONS

$t_D$	Decay time (s)
$\mu_0$	Permeability constant, $4\pi \times 10^{-7}$ H/m
$N_s$	Number of strands [-]
$d_w$	Strand diameter (m)
$V_{ct}$	Volume of composite conductor ( $m^3$ )
$T_{ch}$	Charging time (s)
$L_{cab}$	Cable strand twist pitch (m)
$L_t$	Filament twist pitch (m)
$B_{max}$	Maximum magnetic flux density in the conductor (T)
$B_{ml}$	Field amplitude averaged over the winding section (T)
$a_f$	NbTi filament radius (m)
$B_{c0}$	Minimum penetration field (T)
$J_c$	Current density in superconductor ( $A/m^2$ )
$J_{c0}$	Critical current density at $\theta_0$ and $B_{c0}$ ( $A/m^2$ )
$V_{NbTi}$	Volume of superconductor ( $m^3$ )
$\lambda$	Fraction of superconductor in the composite [-]
$d_{fc}$	Composite filament diameter (m)
$I_0$	Operating current (A)
$I_c(B_{max})$	Critical current at $B_{max}$ (A)
$V_{cc}$	Velocity of the fluid in the central channel (m/s)
$D_{cc}$	Diameter of the central channel (m)
$D_h$	Hydraulic diameter (m)
$P$	Pressure (MPa)
$P_{in}$	Inlet pressure (MPa)
$P_{out}$	Outlet pressure (MPa)
$\Theta$	Temperature of fluid at a pressure P (K)
$\theta_0$	Operating temperature (K)
$\theta_{in}$	Inlet temperature at $P_{in}$ (K)
$\theta_{out}$	Outlet temperature at $P_{out}$ (K)
$h_{overall}$	Overall heat transfer co-efficient ( $W \cdot m^{-2} \cdot K^{-1}$ )
$h_{overall\_UL}$	Overall heat transfer co-efficient per unit length ( $W \cdot m^{-1} \cdot K^{-1}$ )
$h_{conv\_SS}$	Heat transfer co-efficient for flowing helium ( $W \cdot m^{-2} \cdot K^{-1}$ )
$h_{cond\_SS}$	Heat transfer co-efficient for stainless steel tube ( $W \cdot m^{-2} \cdot K^{-1}$ )
$h_{e\_static\_He}$	Heat transfer co-efficient for static helium ( $W \cdot m^{-2} \cdot K^{-1}$ )
$f_f$	Friction factor, 0.045 [-]
$h_{overall\_conv}$	Overall heat transfer co-efficient ( $W \cdot m^{-2} \cdot K^{-1}$ )
$\rho_{He4}$	Density of helium-4 ( $Kg/m^3$ )
$A_{cc}$	Helium flow area in the central channel ( $m^2$ )
$U$	Perimeter for pressure drop (m)
$\Delta p$	Pressure drop per unit length (Pa/m)
$H_{He4}$	Enthalpy of Helium (J/kg)
$\dot{m}$	Mass flow rate (kg/s)
$K_{He}$	Thermal conductivity of helium ( $W \cdot m^{-1} \cdot K^{-1}$ )
$Nu$	Nusselt Number [-]
$Re$	Reynolds Number [-]
$Pr$	Prandtl Number [-]
$\mu_{He}$	Viscosity of helium (Pa.s)
$T_{cond}$	Conductor temperature (K)
$T_m$	Maximum temperature (K)
$\tau$	Coupling current decay time (s)
$J(t)$	Current density at time, t ( $A/m^2$ )
$\gamma$	Density of stabilizer ( $kg/m^3$ )
$\rho(T)$	Resistivity at temperature T ( $\Omega \cdot m$ )
$C(T)$	Heat capacity at T ( $J \cdot kg^{-1} \cdot K^{-1}$ )
$Q_{Tot}$	Total heat load/loss (W)
$MIITs$	Quench load as $10^6 \cdot (Current)^2 \cdot time$

conductor (CICC) using niobium-titanium (NbTi) that provides a particularly simple method for magnet fabrication. The cable technology and winding technology were developed and used to fabricate a first 1.2 m mechanical model of the 3T-SF-CICC dipole. The measurement of cable positions in the winding suggests that the required field homogeneity could be provided by the CICC geometry and dipole structure. As part of the R&D activities, a prototype 1.2 m long dipole magnet is planned to be built and tested in order to evaluate its operational limits and performance parameters. This is a decisive step toward realizing production of other magnets using this NbTi-CICC technology and to guarantee a stable long-term operational performance. Ion beam will be injected to the Ion Ring of JLEIC from a rapid-cycling 8 GeV Booster synchrotron. The preconcept Booster design requires lattice dipoles of 3 T peak field, 1.2 m length, and the same aperture as that required for the arc dipoles of the Ion Ring. Thus, the 3T-SF-CICC dipole design could provide an attractive option for use in the Booster ring as well. The purpose of the study reported here is to evaluate the suitability of the 3T-SF-CICC dipole design for use in the rapid-cycling Booster. Issues of ac losses, conductor stability, and quench protection are considered. Iron is presently not considered as part of the analysis reported except for the calculation of inductance.

Thus, the contribution to the field due to the iron and heating effects within the iron due to fast ramping has not been evaluated. A constant heat load contribution is assumed (*due to the radiation, conduction heat from the cryogenic vessel, as well as iron losses*) to the total loss calculations. An analytical approach is adopted for the study of quench behavior of the 3-T-SF booster model coil and includes ac losses (*due to eddy currents induced in electrically conductive parts*) during fast ramp up or down as well as pressure drop and heating of the coolant flow. These analyses provide a first-order level of understanding of the magnet performance.

A deeper understanding of magnet behavior will be pursued at a later stage using commercially available finite-element analysis codes, e.g., Opera and ANSYS. The analytical model used (Wilson model) provides good insight into the behavior of the magnet using simple and straightforward assumptions under adiabatic conditions [4]–[6]. The symbols and abbreviations used in this article are presented in Table I.

## II. MAGNET DESIGN PARAMETERS

The SF model dipole, 3-T-1.2-m-long prototype-model magnet design proposed by TAMU has two coils (top and bottom), electrically connected in series with each coil utilizing NbTi-CICC. The CICC superconducting windings are supported by a series of G11 plates and surrounded by an iron magnetic core. The basic magnet design parameters are given in Table II.

### A. Conductor

The TAMU-patented [7] CICC consists of 15-strands of NbTi/Cu superconductor, wound onto a central SS perforated tube. This subassembly is then wrapped with SS foil (tape) and

TABLE II  
BOOSTER MAGNET MODEL DIPOLE COIL MAIN PARAMETERS

Parameter	Requirement
Magnet operating current (kA)	13.5
Magnet nominal central Field (T)	3
Magnet inductance, including iron (mH)	0.823
Peak field in the coil (T), <i>with iron</i>	3.3
Booster Magnet Field Ramp rate (T/s)	1.0
Magnetic length (m)	1.2
Beam pipe aperture (mm x mm)	100 x 60
Good field region (mm x mm)	80 x 40
Field Homogeneity (from 0.2 T to 3 T)	$<10^{-4}$
Helium Inlet temperature (K)	4.5

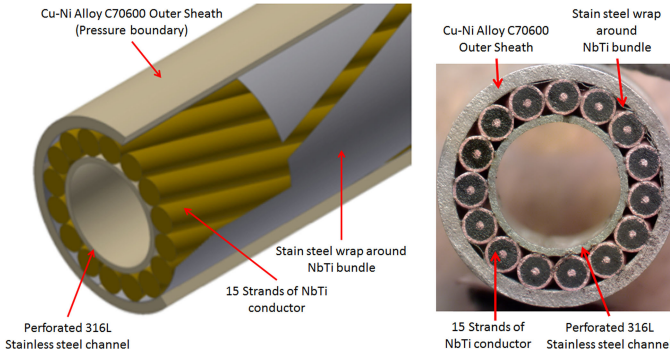


Fig. 2. Schematic and actual cross sectional view of the TAMU CICC.

TABLE III  
SF MODEL DIPOLE CICC PARAMETERS

NbTi/Cu Strands	
Strand Diameter (mm)	1.2±0.0025
Ratio of Cu/Non Cu	1.5
Filament Diameter (μm)	9
# Filaments	>7400
RRR (Residual Resistance ratio)	>150
Filament twist pitch (mm)	18
I <sub>c</sub> (A) at 6.5 T, 4.2 K (Type: VSF-SSCI-ANN)	775
Cable-In-Conduit Conductor	
# Strands	15
Strand twist pitch (mm)	76.6
Outer Tube Material	Cu-Nickel Alloy 70600
Outer Tube dia. - before drawing (mm)	9.52
Outer Tube dia. - after drawing	8.17 ± 0.02
Outer tube wall thickness (mm)	0.5
Inner Tube Material (Perforated)	316L Stainless Steel
Inner tube dia. - after drawing (mm)	4.76 ± 0.02
Inner tube wall thickness (mm)	0.25
Total conductor length for dipole (m)	85 (42.5 m per coil)

inserted into a Cu–Ni sheath tube which is then drawn down to compress the NbTi strands against the central tube to form the completed CICC, as shown in Fig. 2.

The CICC parameters considered for the design and analysis are given in Table III. The stability of the conductor is evaluated against the design cycle of ac losses and availability of cooling power of pressurized helium flowing in the central channel of the CICC and is presented later in the following section.

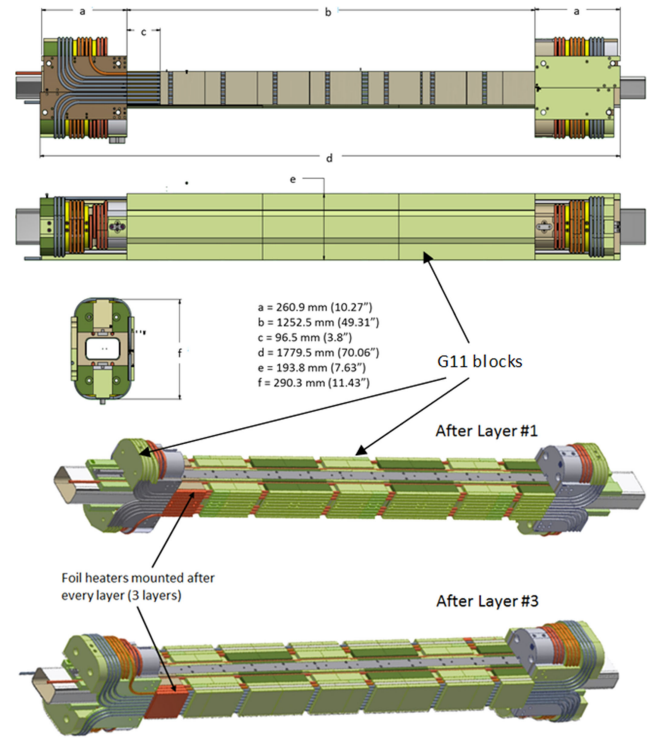


Fig. 3. Typical overall dimensions of the 1.2 m dipole showing the foil heater mounting locations after every layer.

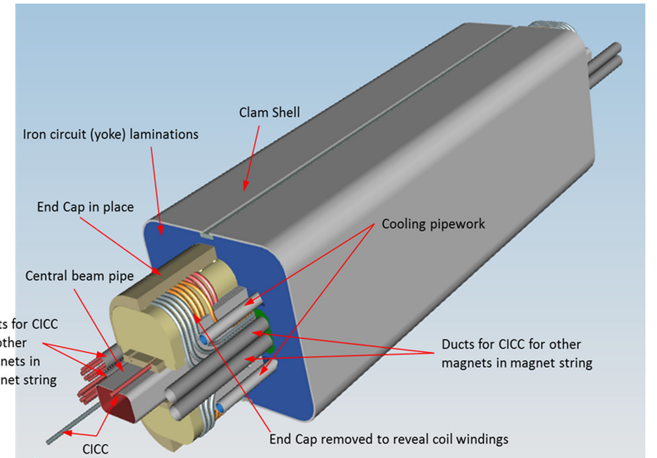
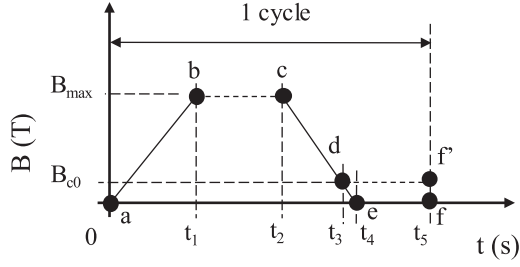


Fig. 4. Schematic of model dipole—cold mass assembly.

### B. Cold Mass Assembly

The model SF dipole has a total of 24 turns (*12 turns per coil*), and a total cable length of about 85 m. The NbTi cable is cooled by helium flowing through the central tube. As the coolant flows through the CICC, its pressure will drop. In order to avoid bubble formation, the helium needs to be kept in the super-critical state throughout the winding pack. For each coil the individual coil turns are held in place using a series of G11 blocks and plates (as shown in Fig. 3). A laminated iron magnetic core is fitted around the CICC winding pack. The iron core also has independent cooling tubes located within its structure. A typical model of the cold mass assembly is shown in Fig. 4.





- a-b: magnet ramp-up to maximum field  
b-c: magnet parked at field  
c-d, d-e: magnet ramp-down to a minimum or zero field  
d-f: Magnet parked at a minimum field  
e-f: Magnet at zero field  
 $t_5$ : One cycle time period  
 $0 - t_1$ : Ramp up time (s)  
 $t_2 - t_3$  or  $t_2 - t_4$ : Ramp down time (s)  
 $t_1 - t_2$ ,  $t_3 - t_5$  and  $t_4 - t_5$ : Time to be defined based on Physics requirements

Fig. 5. Schematic representation of the ac loss cycle used for analysis during magnet ramping.

The main advantage of using CICC is the manufacturing/fabrication of CICC and winding on the main support structure for the coils, which is based on the design and associated bending tooling developed at TAMU. The development is envisaged to have a positive cost impact compared to a conventional Rutherford cable based on  $\cos(n\theta)$  magnets.

### III. SF MODEL DIPOLE ANALYSIS

The challenges associated with this fast ramping dipole design include risks associated primarily with eddy current effects, impact on the cryogenics, magnet stability, and quench protection, which were analyzed using approaches described in [4]–[6] and [8]–[10]. Critical current for the conductor was established using Bottura's model [11].

#### A. AC Losses

Due to fast current ramping (varying field), eddy currents are induced in the conductor (wire and cable), in the iron and within the electrically conductive structural elements of the magnet. This creates ac losses under continuous operation while running the magnet up and down based on the cycle, as shown in Fig. 5. The losses in the CICC should be minimized by appropriate selection of conductor, while the magnet conductor cooling needs to be capable of extracting the heat loads during operation. The ac loss components are calculated based on the Wilson model [4], [5] operating at the parameters indicated in Table II [12]–[14]. The losses in the SS foil and in the Copper–Nickel (CuNi) outer sheet have not been considered for the analysis, here, as the SS and Cu–Ni have high electrical resistivity, and ac loss in the NbTi is considerably more prominent. In practice, the Cu–Ni will improve the thermal capacity and stability of the conductor.

The ac loss calculations are based on a loss cycle as represented in Fig. 5. The analysis presented is based on the time to reach full field,  $B_{\max}$  in time  $t_1$  as a worst-case scenario. The

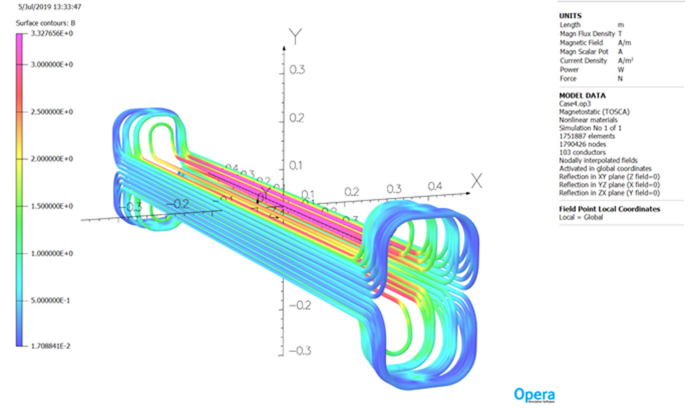


Fig. 6. Magnet field distribution over the conductor ( $B_{\max} = 3.328$  T).

losses are calculated for a half-cycle based on the assumption,  $t_2 = t_1$  and  $t_5 = t_4$  (actual time to be defined per physics requirements) making the one-cycle period as ramp up to full field and down to zero. Therefore, the power losses are calculated based on the energy loss in a half-cycle in a given time period ( $t_1$  is 3 or 12 s for a ramp rate of 1 or 0.25 T/s, respectively). The magnetic flux density calculated using Opera finite element analysis (FEA), as shown in Fig. 6, varies between 3.33 and 0.02 T at an operating current of 13.5 kA without the iron circuit (3 T at magnet center).

The ac loss components are calculated as follows:

1) *Eddy Current/Coupling Losses ( $E_{EC}$ )*: Calculated losses in the strands are the coupling losses between filaments for a half-cycle (either current ramp up or ramp down). The following equation suggests that  $\tau$  needs to be decreased (by increasing the resistance of the material around the filament) in order to reduce the eddy current loss in the composite filamentary conductor:

$$E_{EC} = B_{ml}^2 \cdot \frac{8\tau}{2\mu_0 T_{ch}} \cdot V_{ct} \cdot \left\{ 1 - \frac{\tau \cdot (1 - e^{-\frac{T_{ch}}{\tau}})}{T_{ch}} \right\}. \quad (1)$$

The local field amplitude  $B_{ml}$  is approximated magnetic field (flux density) amplitude averaged over the winding section as  $B_{ml} = \frac{B_{\max}}{\sqrt{3}}$ .

2) *Hysteresis Losses ( $E_{HY}$ )*: This is calculated for a half-cycle (ramp up or down). The magnetic field in the coil is ramped down to a minimum equivalent to the self-field associated with the ratio  $I_0/I_C(B_{\max})$  for minimum field penetration. This is to an approximation that field varies linearly over the winding and varies between minimum field penetration to maximum field, occupying an equal volume of winding. Therefore, averaged value of hysteresis loss is calculated as

$$E_{HY} = \frac{8}{3\pi} \cdot a_f \cdot J_{c0} \cdot B_{c0} \cdot \left\{ \frac{B_{\max} + B_{c0}}{B_{\max}} \cdot \ln \left( \frac{B_{\max} + B_{c0}}{B_{c0}} \right) - 1 \right\} \cdot V_{NbTi}. \quad (2)$$

TABLE IV  
RESULTS OF SF MODEL COIL AC LOSSES VERSUS RAMP RATES

Loss Component	Ramp rates	1 T/s	0.5 T/s	0.25 T/s
	Charging time to reach full field	3 s	6 s	12 s
Eddy current (coupling and Magnetization) loss – filaments and strands (J)	Induced currents between SC filaments due to external field changes and between strands	79.12	39.56	19.78
Hysteretic loss (J)	Induced currents within SC filaments	8.97	8.97	8.97
Penetration loss (J)	Superconductor surface	1.77	1.77	1.77
Self-field loss (J)	Induced currents between SC filaments due to changes in the transport current	3.68	3.68	3.68
<b>TOTAL AC LOSS, <math>E_{\text{Tot\_ac}}</math> (J)</b>		<b>93.54</b>	<b>53.98</b>	<b>34.20</b>
<b>TOTAL AC LOSS, <math>Q_{\text{Tot\_ac}}</math> (W) – Only during ramp</b>		<b>31.18</b>	<b>9.00</b>	<b>2.85</b>
<b>TOTAL LOSS, <math>Q_{\text{Tot}}</math> (W) (Includes a constant 4 W heat load)</b>		<b>35.18</b>	<b>13.00</b>	<b>6.85</b>

where

$$B_{c0} = B_p F_{\text{SF}}, \quad F_{\text{SF}} = I_0 / I_C (B_{\text{max}}), \quad B_p = \mu_0 \lambda J_c \frac{d_{\text{fc}}}{2}.$$

3) *Penetration Losses  $E_P$* : For a fast-ramping magnet, we cannot neglect the effect of field penetration also sometimes referred to as skin effect, where the current flows on the outer filaments surface and occupies a finite volume. The penetration losses in filamentary wires at  $B_{\text{max}}$  are calculated for a composite filament diameter in a fast-changing longitudinal magnetic field. The losses are calculated for a half-cycle as

$$E_P = \frac{1}{2} \left( \frac{B_{\text{max}}^2}{2\mu_0} \right) \frac{4\pi^2 \cdot \left( \frac{d_{\text{fc}}}{2} \right)^2}{L_t^2} Y(X) V_{\text{NbTi}}. \quad (3)$$

where

$$Y(X) = \frac{X}{3(1+X)^2} \quad \text{if } X < 1;$$

$$= \frac{1}{2X} - \frac{5}{12X^2} \quad \text{if } X > 1, \quad \text{and } X = \frac{2\pi B_{\text{max}}}{\mu_0 \lambda J_{c0} L_t}.$$

4) *Self-Field Losses  $E_{\text{SF}}$* : This is associated with the loss effects due to the self-field of the wire from transport current

$$E_{\text{SF}} = \frac{(B_p F_{\text{SF}})^2}{2\mu_0} M(F_{\text{SF}}) V_{\text{NbTi}} \quad (4)$$

$$M(F_{\text{SF}}) = \left[ \frac{4}{F_{\text{SF}}} - 1 + \frac{4(2 - F_{\text{SF}}) \ln \left( \frac{2 - F_{\text{SF}}}{2} \right)}{F_{\text{SF}}^2} \right]. \quad (5)$$

A summary of ac losses calculated with varying ramp rates is given in Table IV. Other external heat load has been estimated at a constant 4 W (due to the radiation, conduction heat from the cryogenic vessel, as well as iron losses) [15]. Detailed calculations for the radiation and iron losses shall be carried out and engineered during the design of the iron magnetic circuit and cryostat structures. The major loss components are estimated at three different ramp rates for comparison.

TABLE V  
SUMMARY OF AC LOSSES WITH TIGHTER FILAMENT TWIST PITCH (7.1 mm)

Loss Component	Ramp rates	1 T/s	0.5 T/s	0.25 T/s
	Charging time to reach full field	3 s	6 s	12 s
Eddy current (coupling and Magnetization) loss – filaments and strands (J)	Induced currents between SC filaments due to external field changes and between strands	12.98	6.49	3.25
Hysteretic loss (J)	Induced currents within SC filaments	8.97	8.97	8.97
Penetration loss (J)	Superconductor surface	21.38	21.38	21.38
Self-field loss (J)	Induced currents between SC filaments due to changes in the transport current	3.68	3.68	3.68
<b>TOTAL AC LOSS, <math>E_{\text{Tot\_ac}}</math> (J)</b>		<b>47.01</b>	<b>40.53</b>	<b>37.28</b>
<b>TOTAL AC LOSS, <math>Q_{\text{Tot\_ac}}</math> (W) – Only during ramp</b>		<b>15.67</b>	<b>6.76</b>	<b>3.11</b>
<b>TOTAL LOSS, <math>Q_{\text{Tot}}</math> (W) (Includes a constant 4 W heat load)</b>		<b>19.67</b>	<b>10.76</b>	<b>7.11</b>

Losses were recalculated with using a tighter twist pitch (7.10 mm, instead of 18 mm) which suggests that the coupling and magnetization losses in filaments and strands can be reduced by a factor of about 6 while the penetration losses increase by a factor of about 12. The ac losses are primarily dominated by magnetization losses in filaments and strands, thus the total ac loss ( $E_{\text{Tot\_ac}}$ ) decreases by a factor of about 2 at the higher ramp rate of 1 T/s and increases by about 10% at 0.25 T/s, as summarized in Table V.

## B. Cooling and Temperature Margin

Appropriate design of cryogenics is critical for the stable operation of the magnet. The heat load dynamics due to ac loss is based on the ramp rate, field, and operating cycles per unit time. Therefore, it is critical to optimize the helium flow rate and pressure drop in the cable to allow for removal of the steady state ac losses caused by the fast ramp (1.0 T/s). A basic design temperature stability margin of 1 K for normal operation is assumed [16]. Pressure drop and temperature rise analysis are carried out to define the input pressure and flow rate limits [17]–[19]. A friction factor  $f_f$  of 0.045 is used for the central CICC channel throughout the analysis as measured by Chavez [20]. The pressure drop calculations are performed using the standard Darcy–Weisbach friction loss equation (and compared with the ITER conductor) [21], [22]. The helium outlet temperature is calculated based on the heat load balance over a specific length of the conductor. This estimate is primarily based on the difference between the amount of heat deposited and heat removed by helium within the conductor

$$\Delta p(V_{\text{cc}}, D_{\text{cc}}, P, \theta) = f_f \times \frac{1}{D_h(D_{\text{cc}})} \times \frac{\rho_{\text{He4}}(P, \theta) \times V_{\text{cc}}^2}{2} \quad (6)$$

where  $D_h(D_{\text{cc}}) = 4 \times \frac{A_{\text{cc}}(V_{\text{cc}})}{U(D_{\text{cc}})}$ ,  $A_{\text{cc}}(V_{\text{cc}}) = \pi \cdot \left( \frac{D_{\text{cc}}^2}{4} \right)$ , and  $(D_{\text{cc}}) = \pi \cdot D_{\text{cc}}$ .

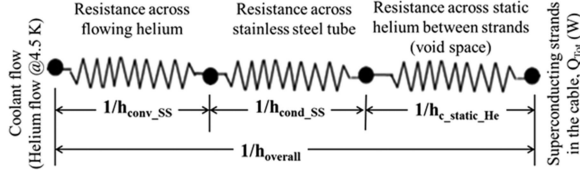


Fig. 7. Equivalent thermal resistance model showing the overall heat transfer resistance for extraction of heat from the superconductor by helium.

The  $\Delta p$  is calculated after every turn [using the function defined above in (6)]; this sets the inlet pressure for the subsequent turn and is integrated over the length in order to evaluate the overall  $\Delta p$ . In order to evaluate the outlet temperature of helium, a constant heat load (summarized in Table IV) distributed over the length of the conductor is used. An energy balance, utilizing the enthalpy of the flowing helium in the central channel of the CICC to extract the heat has been developed to produce stable operation. All standard thermo-physical properties of helium used in the calculations have been obtained from the NIST database [23].

$Q_{\text{Tot}} = \dot{m} \times [H_{\text{He4}}(P_{\text{in}}, \theta_{\text{in}}) - H_{\text{He4}}(P_{\text{out}}, \theta_{\text{out}})]$ , based on the energy balance equation,  $\theta_{\text{out}}$  is calculated as a function of  $Q_{\text{Tot}}$ ,  $P_{\text{in}}$ ,  $P_{\text{out}}$ ,  $\theta_{\text{in}}$ , and  $\dot{m}$ . The effective thermal resistance for the heat extraction by helium from the superconductor (under normal operation) includes the following.

- 1) Flowing helium inside the SS tube (based on the Dittus–Boelter equation)

$$h_{\text{conv\_SS}}(V_{\text{cc}}, D_{\text{cc}}, P, \theta) = K_{\text{He}}(P, \theta) \times \frac{Nu(V_{\text{cc}}, D_{\text{cc}}, P, \theta)}{D_{\text{cc}}} \quad (7)$$

where

$$Nu(V_{\text{cc}}, D_{\text{cc}}, P, \theta) = 0.023 \times \text{Re}(V_{\text{cc}}, D_{\text{cc}}, P, \theta)^{0.8} \times \text{Pr}(P, \theta)^{0.4} \quad (8)$$

$$\text{Re}(V_{\text{cc}}, D_{\text{cc}}, P, \theta) = \frac{\rho_{\text{He4}}(P, \theta) \cdot V_{\text{cc}} \cdot D_{\text{h}}(D_{\text{cc}})}{\mu_{\text{He}}(P, \theta)} \quad (9)$$

- 2) Across the central perforated SS tube thickness.
- 3) Across the static helium filled voids between the strands.

The overall heat transfer coefficient is calculated using the three heat transfer coefficient components based on an equivalent thermal resistance model, as shown in Fig. 7, and subsequently defined for a unit length

$$h_{\text{overall\_UL}}(V_{\text{cc}}, D_{\text{cc}}, P, \theta) = h_{\text{overall}}(V_{\text{cc}}, D_{\text{cc}}, P, \theta) \times \pi D_{\text{cc}} \quad (10)$$

where

$$h_{\text{overall}}(V_{\text{cc}}, D_{\text{cc}}, P, \theta) = \left[ (h_{\text{conv\_SS}}(V_{\text{cc}}, D_{\text{cc}}, P, \theta))^{-1} + (h_{\text{c\_static\_He}}(P, \theta))^{-1} + (h_{\text{cond\_SS}}(\theta))^{-1} \right]^{-1} \quad (11)$$

TABLE VI  
SUMMARY OF HELIUM PARAMETERS VERSUS RAMP RATES

Series Cryogenic Cooling Circuit (85 m for both coils)			
Parameter	1.0 T/s	0.5 T/s	0.25 T/s
Helium flow rate (g/s)	2.0	2.0	2.0
Inlet Pressure of Helium (bar)	2.5	2.5	2.5
Helium pressure drop (bar)	0.71	0.71	0.71
Helium inlet temperature (K)	4.5	4.5	4.5
Max. conductor temperature (K)	5.434	5.013	4.919
Max. CICC temperature rise (K)	0.934	0.513	0.419

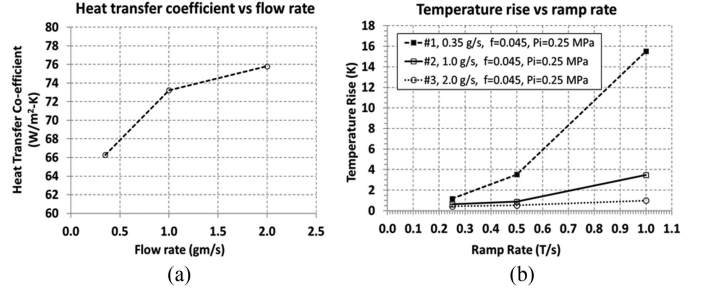


Fig. 8. (a) Estimation of overall heat transfer coefficient versus helium flow rate. (b) Estimation of the temperature rise with ramp rate.

Summary of helium flow rates (with coolant pressure drops) required to limit the temperature rise, not higher than 1 K, at varying ramp rates is given in Table VI. The variations of the overall heat transfer and temperature rise are shown in Fig. 8(a) and (b) for helium flow rates between 0.35 and 2.0 g/s. The magnitude of the heat transfer coefficient tends to plateau at higher flow rates. Therefore, the present analysis has been limited to a flow rate of 2.0 g/s. Analysis shows the temperature rise can be limited to 1 K at 1.0 T/s with 2.0 g/s at 2.5 bar inlet pressure. This suggests that the temperature rise can be managed with increased helium flow to a level before the heat transfer coefficient plateaus and limits the heat extraction capability. The pressure drop is not insignificant but is manageable.

### C. Model Equivalent Single Coil Quench Analysis

The stored energy of the model 3 T-SF dipole is  $\sim 75$  kJ and a dump resistor of  $0.05 \Omega$  is used to extract the energy externally. An equivalent single coil Wilson model [6] is used for a preliminary quench calculation.

**Quench Integral:** The quench integral or quench load (also sometimes referred to as MIITs) is evaluated based on the Wilson model [6] as indicated below for a worst-case scenario in the event of a quench, where the current density remains constant at the initial value for the whole decay time. The MIITs calculation uses only the material properties of the conductor to evaluate the time required to reach a certain temperature. The user then has to use this information to establish the time interval to reach a “safe” temperature for the weakest point in the magnet (for example, the melting point of any solder used for making conductor joints)

$$\int J(t)^2 dt = \int_{4.7\text{K}}^{T_m} \frac{\gamma \cdot C(T_{\text{cond}})}{\rho(T_{\text{cond}})} dT_{\text{cond}} \quad (12)$$

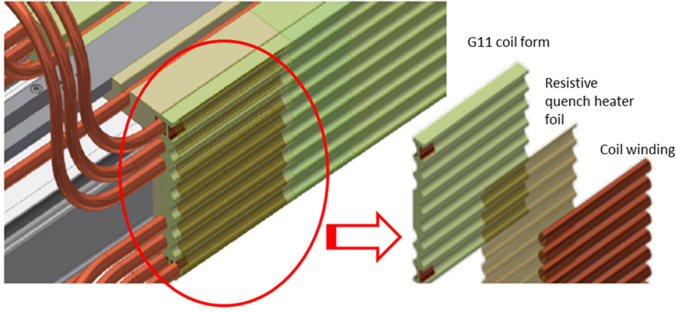


Fig. 9. Schematic representation of quench heater location and mounting.

TABLE VII  
BRIEF SUMMARY OF CALCULATED CRITICAL PARAMETERS

Parameter (calculated)	Case#1	Case#2
Operating temperature (K)	4.92	
Current sharing temperature (K)	6.97	
Temperature margin (K)	2.05	
Short sample performance (%)	61.9	
MPZ (mm)	0.16	
MQE (mJ)	3.79	
Conductor length used for quench calculation (m)	42.5	3.5
Hot spot temp. (K) / Time to reach the corresponding hot spot temperature (ms)	<i>Initial</i>	53.9/21.7
	<i>Final</i>	70.9/ 39.81
Max. voltage, Line to Ground (kV)	> 2.5	1.3
Max. MIITs at 200 K ( $10^6 \cdot A^2 \cdot s$ )	-	16.2
MIITs estimated with dump resistor ( $10^6 \cdot A^2 \cdot s$ )	-	1.19
Time required to run the magnet to 0 A incl. detection time (ms) for design	-	< 49
†unsafe leading to magnet damage		

TABLE VIII  
SUMMARY OF SF MODEL DIPOLE STABILITY

Parameters evaluated	Passed	Remarks
Short sample performance (SSP) in %	Yes	< 75 (†62.8)
Temperature margin (Sharing temperature) K	Yes	> 1.5 (†1.97)
Stable for Beta (Adiabatic stability)	Yes	
Adiabatic flux jump stability	Yes	
Dynamic stability	Yes	
Filament twist pitch stability	No	need < 7.15 mm
Stable for finite element size	Yes	
Cryogenic Stability	*	Not for the CICC
†calculated values		

Two scenarios were investigated to evaluate the required magnet protection. Assumption, the quench would start from one spot and that spot gets continuously heated under adiabatic conditions. MIITs are evaluated both with dump resistor and without the dump resistor (all energy dumped in the coil) in the following cases.

- 1) Case#1: Model dipole without any active quench heaters, with the whole conductor length active in each half of the dipole (42.5 m).
- 2) Case#2: Model dipole with active quench heaters on one side of both top and bottom coils. Assuming an active conductor length of approximately 3.5 m each turn of one half of the dipole as an effective length of conductor

TABLE IX  
LEVEL WITH RANKING OF SEVERITY

Value	Effect	Severity
1	No Impact	No impact to fit, form, or function.
2	Negligible	Potential for impact on appearance or an annoyance if a component, system, subsystem, or assembly does not meet design requirements.
3	Marginal	Potential for a moderate impact or loss of the secondary function of a component, system, subsystem, or assembly if it does not meet design requirements.
4	Significant	Potential for an adverse impact or degradation of primary function if a component, system, subsystem, or assembly does not meet design requirements.
5	Critical	Potential for serious impact or loss of primary function of a component, system, subsystem, or assembly if it does not meet design requirements.
6	Crisis	Potential for violation of a regulation or catastrophic failure during operation if the component, system, subsystem, or assembly does not meet design requirements

TABLE X  
LEVEL WITH RANKING AND PROBABILITY OF OCCURRENCE

Value	Effect	Occurrence Risk	Probability
1	Improbable	Few failures with identical design.	Occurring not more than once in its life time.
2	Remote	Few failures with similar design.	Occurring not more than once in ten years.
3	Occasional	Frequent failures of similar designs, or failure not likely with new design or application.	Occurring not more than once in five years.
4	Probable	Failures expected with new design or application.	Could occur once in two years.
5	Frequent	Failures inevitable or new design without precedent.	Could occur annually.

TABLE XI  
LEVEL WITH RANKING AND PROBABILITY OF DETECTION

Value	Detection Likelihood	Detection (Design Controls)	Detection (Operational Design Controls)
1	Almost Certain	Failure prevented through proven design solution.	Failure mode almost certainly detected by continuous automatic online testing.
2	High	Analysis with strong detection capability and high correlation to operating conditions.	Failure mode likely detected by continuous automatic online testing.
3	Medium	Product validation testing prior to design freeze.	Failure mode likely detected by periodic online testing/monitoring.
4	Low	Product validation testing prior to launch, but after design freeze.	Failure mode likely detected by periodic offline testing or inspection.
5	Very Low	Analysis with weak detection capability or low correlation to expected operating conditions.	Failure mode only likely to be detected by unplanned testing or inspection.
6	Remote	No design controls in place.	Failure mode not detectable in service.

(considering all turns are of equal length for the analysis), as shown in Fig. 3.

A resistive quench heater foil is located at one end of the body length of the dipole, as shown in Fig. 9. Each serpentine shaped quench heater foil is in thermal contact with each turn of the



TABLE XII  
EXTRACT FROM THE FMEA WORKSHEET FOR THE FAST RAMPING 3-T-SF BOOSTER MAGNET

FMEA WORKSHEET - Failure Mode & Effects Analysis										
Potential Failure Mode	Potential Effects of Failure	S	Potential Causes	O	D (BM)	RPN-BM	Controls could be in place	D	Recommended Actions	RPN-AM
<i>Conductor:</i> Design (Structural, Pressure)	Conductor overpressure and burst	6	Overpressure during quench event	4	3	72	Adequate design analysis and safety design with reliefs as per ASME and JLab standard	1	Adequate design to accommodate the quench scenario with all energy dumped in the conductor	24
<i>Conductor:</i> AC losses for fast ramping magnets	Increased Heat load on the conductor and magnet during ramping	4	Filament size and twist pitch of Sc filament with the Base material matrix of the strands	3	3	36	Appropriate design analysis carried out using published data	1	Appropriate selection of conductor with proper filament size and twist pitch	12
<i>Conductor:</i> Stability	Increased Heat load and hot spot temperature during ramping and quench	6	Not enough thermal stabilizer and improper design (magnet, bus-lead, splice), burnout	4	2	48	Adequate design analysis and reviews (design stage)	1	Adequate design to accommodate the quench scenario	24
<i>Conductor:</i> Cooling mechanism inadequate	Conductor at elevated temperature	5	Limited flow, high heat load, high ac losses during ramping, yoke at elevated temperature	4	3	60	develop the operating boundary and predict with FEA & analytical models	1	Adequate selection of Flow, pressure and controlled ramp	20
<i>Conductor:</i> Burn out of the conductor in case of a quench	Inadequate cooling and hot spot temperature too high to handle and propagate	5	inadequate design of the magnet and protection circuit	4	4	80	Adequate protection design analysis (design stage)	1	Design and test to define a reference or adopt any reference design used in the past	20
<i>Magnet:</i> Could not make 1 T/s	Effect the performance of the magnet requirement	4	Inadequate conductor, cooling capacity and heat load, others	4	4	64	Design and test R&D prior to actual magnet design and built for the system	2	Design and test to define the boundary condition	32
<i>Magnet:</i> Coil cannot accommodate high flow rates	Reduced cooling and reduced temperature margin	4	Cryogenic limitation and block in the cooling path/high heat load	4	3	48	develop the operating boundary and predict with FEA & analytical models	1	Adequate selection of Flow, pressure and controlled ramp (determine the ramp rate envelope)	16

SEVERITY (S) = How severe is effect on the customer; OCCURANCE (O) = How often or frequent; DETECTION (D) = How probable is detection of cause; RPN = S \* O \* D; RPN-BM is RPN before mitigation; RPN-AM is RPN after mitigation; D (BM) = Detection (before mitigation); Maximum RPN = 6 (S) \* 5 (O) \* 6 (D) = 180.

CICC for both the top and bottom coils (length of conductor evaluated is about 42.6 m/12 turns per dipole = 3.5 m). A summary of calculated critical parameters are given in Table VII. Characteristic parameters, such as minimum quench energy (MQE) and length of minimum propagation zone (MPZ), have been calculated for future reference, and will be required for the design of splices and coil interconnects/bus bars and overall magnet stability.

Preliminary analysis for Case#1 (with conductor length 42.5 m in one-coil of the dipole) suggests that an excessive temperature rise will occur at the point of quench initiation (also referred to as the *hot spot temperature*), with potential damage to the coil. Case #2 suggests a hot spot temperature of ~140 K in 65 ms, with MIITs limited to ~16.2 MA<sup>2</sup>s for a 200 K temperature rise without a dump resistor and 1.19 MA<sup>2</sup>s with a dump resistor suggesting that the magnet should be safe. The quench heater foil (used in case #2) is assumed to be an ideal heater that can transfer the heat instantaneously to the conductor; therefore, for future studies a thermal time constant will need to be added to the overall time. The analysis also suggests that the time required to run the magnet down (which includes the quench detection time) should be less than 49 ms, which is an extremely challenging requirement. Therefore, fast active protection is one critical requirement using fast electronics to sense and interrupt the current from the power supply to ensure the safety of the magnet.

#### D. Magnet Stability

The model dipole was also evaluated for stability with the given conductor under static conditions at full operating current using multiple stability criteria [4], [24], [25]. The summary of the stability evaluation is given in Table VIII.

The SSP and the temperature margin suggest that magnet is stable but will require a tighter filament twist pitch for the overall magnet stability. One perceived benefit having CICC is having a direct contact between the superconducting strands and supercritical helium is envisaged to dissipate microquenches before they propagate [8].

#### IV. RISK MITIGATION APPROACH

At the present stage, this design has been subjected to a failure modes and effects analysis (FMEA) process, limited to failure mitigation from a technical perspective, driving designs that will achieve safe operation of the magnet system based on the experience gained from the 12 GeV upgrade project at JLab [26]. The primary focus has been on the protection system and its adequacy for the proposed magnet design.

JLab utilized the FMEA methodology to identify the highest risk aspects of the CICC. This article identifies the first series of analyses performed in response to the identified risks, via ranked risk priority number (RPN). Due to the requirement that accelerator magnets exhibit design robustness, it is critical to



fully validate the CICC design, refining as necessary, in order to be seen as equivalent in robustness to Rutherford cable.

The FMEA is applied using the RPN *criteria*—potential failure modes, potential effects of failure, potential causes, and controls that could be in place with recommended actions. The RPN is evaluated based on the severity (S), occurrence (O), and detection (D) level/ranking, given in Tables IX, X, and XI, respectively. The following *entities* have presently been evaluated based on the RPN—splices, conductor, magnet, support structure, electrical failure, vacuum vessel and pumping system, control system, steel, and beam-chamber during the preliminary design. A detection probability was generated based on the likelihood of mitigation before the issue became a failure or detection before it has a significant consequence. An RPN after mitigation (RPN-AM) was generated from the product of the three values S, O, and D. A sample of the identified failure modes are given in Table XII for the conductor and magnet. The failure modes were grouped into three levels by these RPNs: low risk  $<18$ , moderate risk  $<45$ , and high risk  $\geq 45$ . FMEA exercise suggests that further studies, analyses, short sample testing, and prototype program are necessary to demonstrate the robustness for CICC and the magnet.

## V. CONCLUSION

In order to establish the performance with the new conductor and to provide guidelines for testing, preliminary analyses of the 3-T-SF model dipole have been carried out in order to set limits for the key operational parameters (cryogenics and electromagnetic) in terms of magnet stability during a fast ramp. The analyses have demonstrated a few limitations with the superconducting strands (in terms of instability), with filament twist pitch and impact on cryogenics (helium flow rate) for a fast ramping magnet (1 T/s). The ac losses can be reduced significantly and the effect can be mitigated with a tighter twist pitch and sizing of filaments together with proper selection of matrix material with improved thermal capacity. This will also help on the development of fast active protection to bring the magnet current down without damaging the magnet. It is critical to further identify the risks and have a full program of refining the design for FMEA with a complementary design validation, including testing of the CICC as well as full scale magnets.

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